

(Microreactor Applications Research, Validation & EvaLuation),

Preliminary MARVEL Accident Scenarios for Heat Extraction System

May 18-20, 2026

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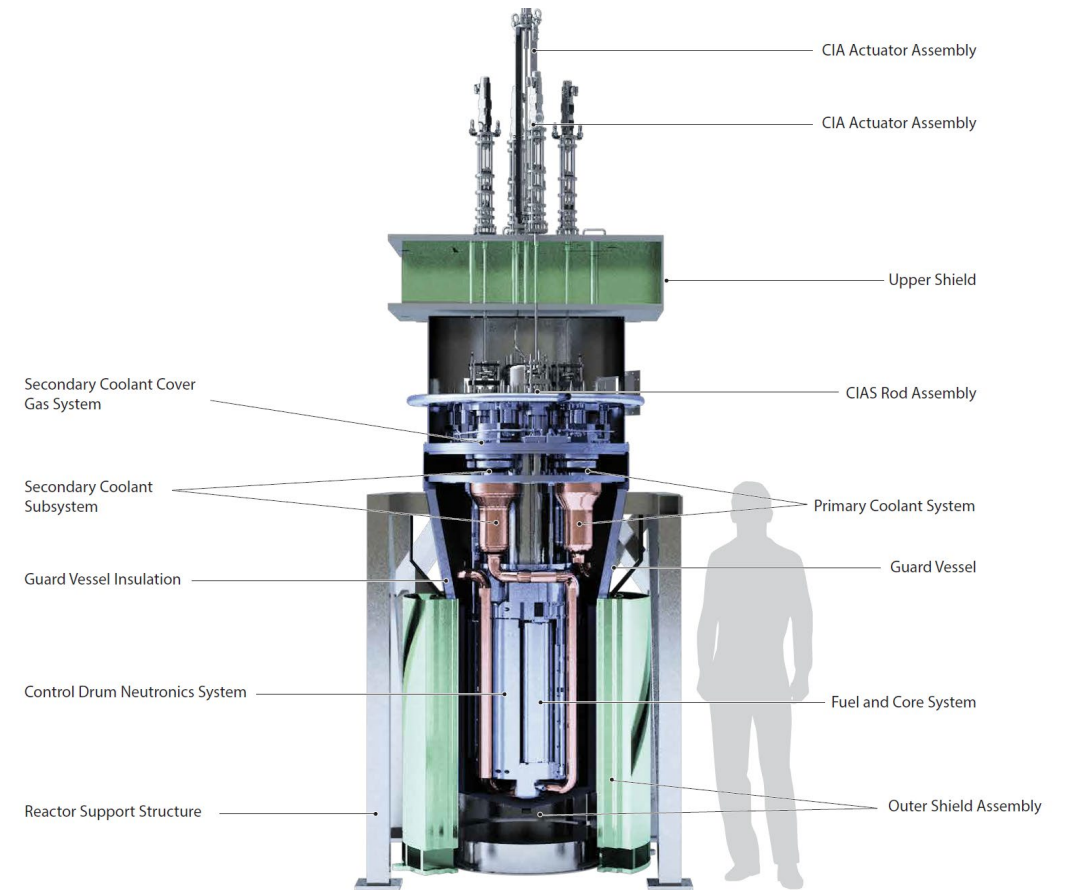
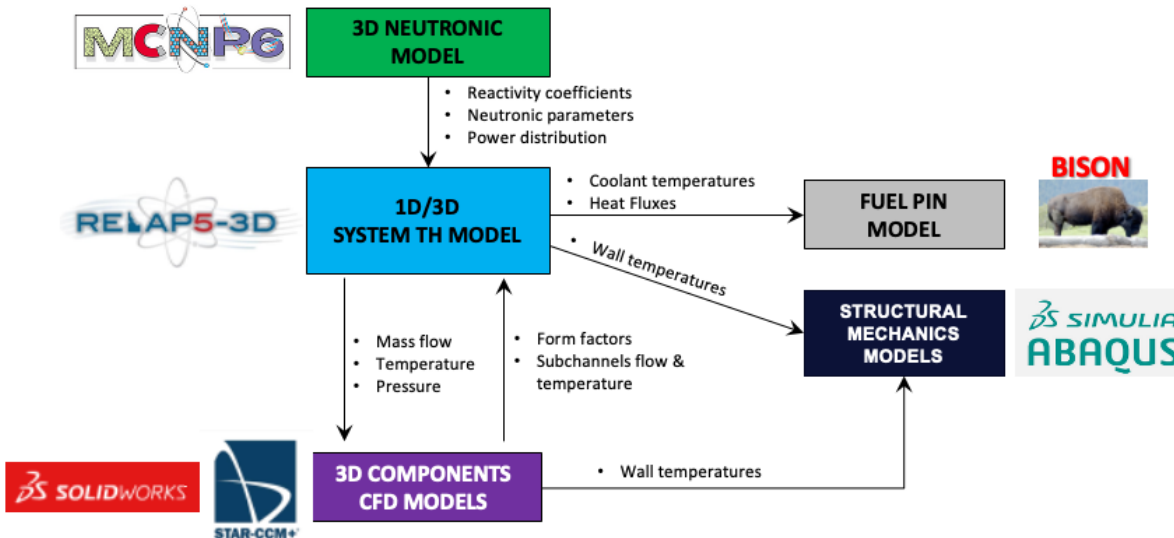
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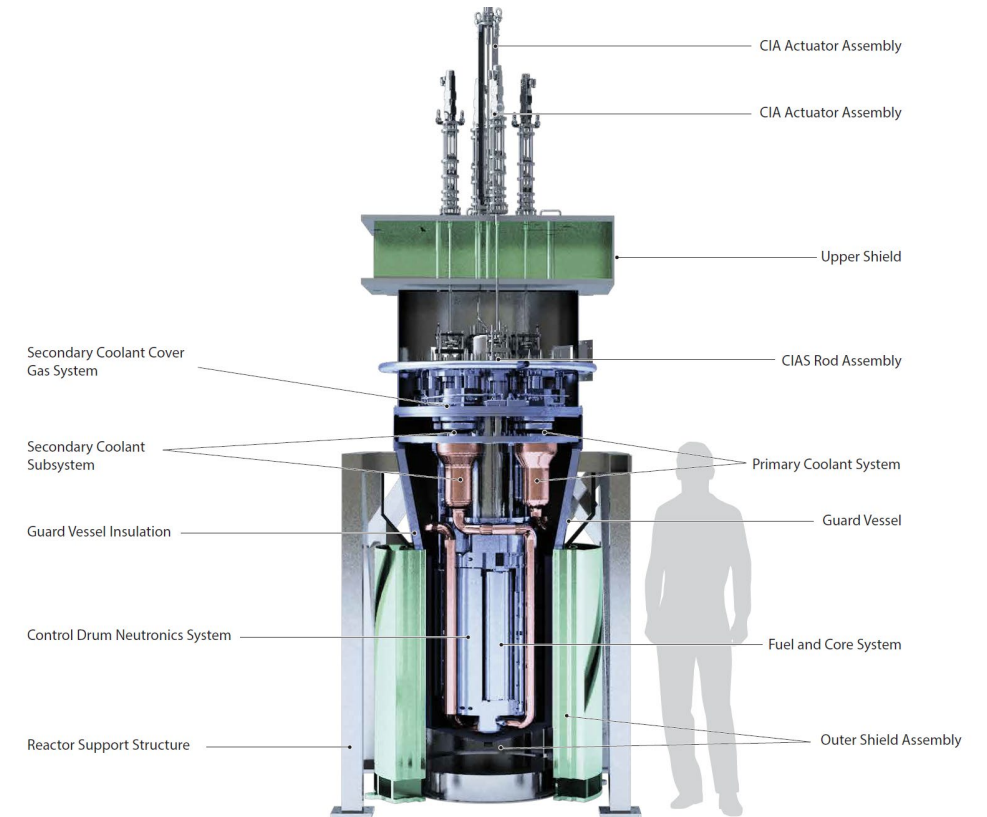
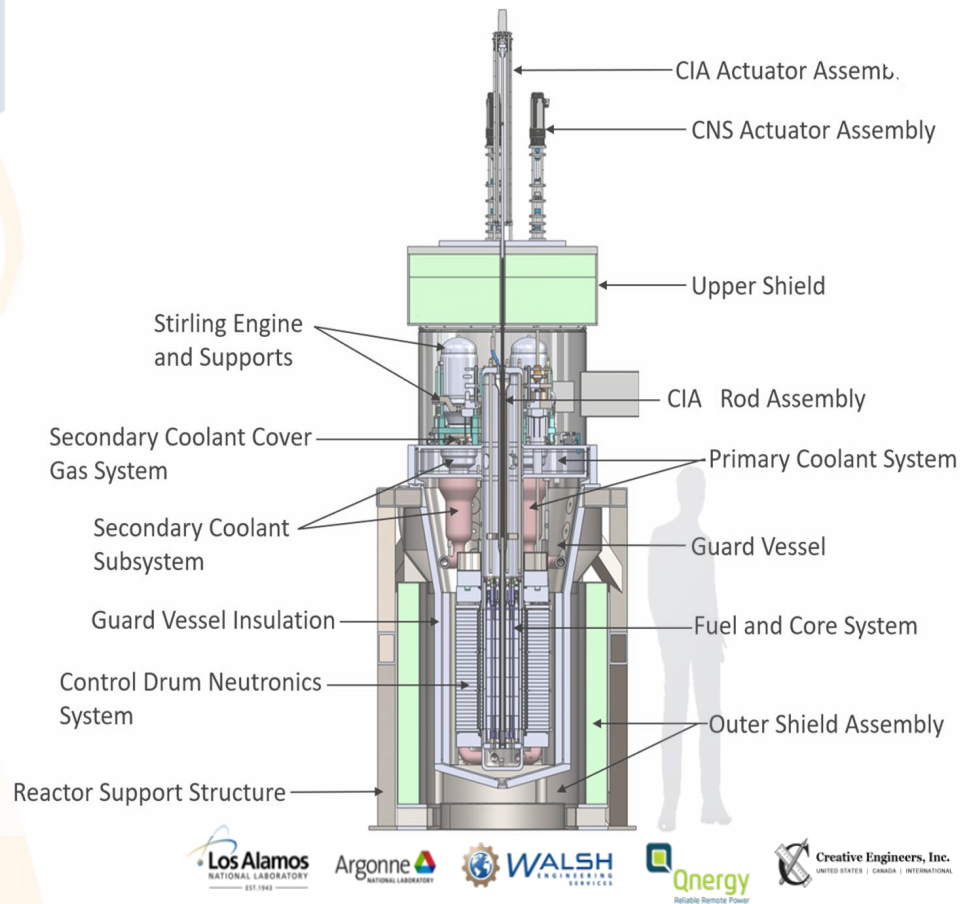


MARVEL Project aims to design, deploy, and demonstrate a microreactor

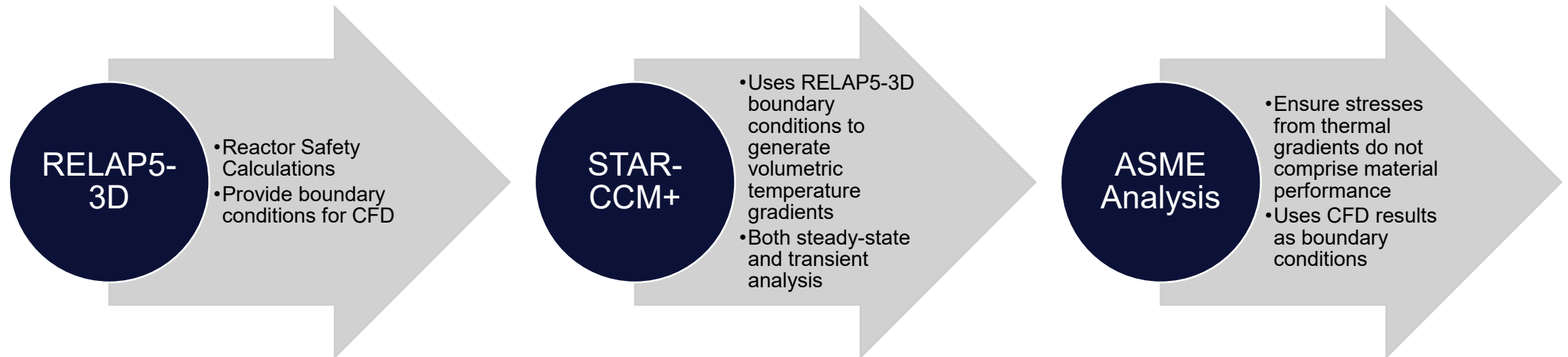
- Identify challenges industry designers will encounter before they do so we can help brainstorm solutions
- Provide stakeholders outside of nuclear with the opportunity to engage with a real reactor prior to making decisions on how to use nuclear for their needs



Original MARVEL design used Stirling engines connected to the intermediate heat exchanger (IHX)



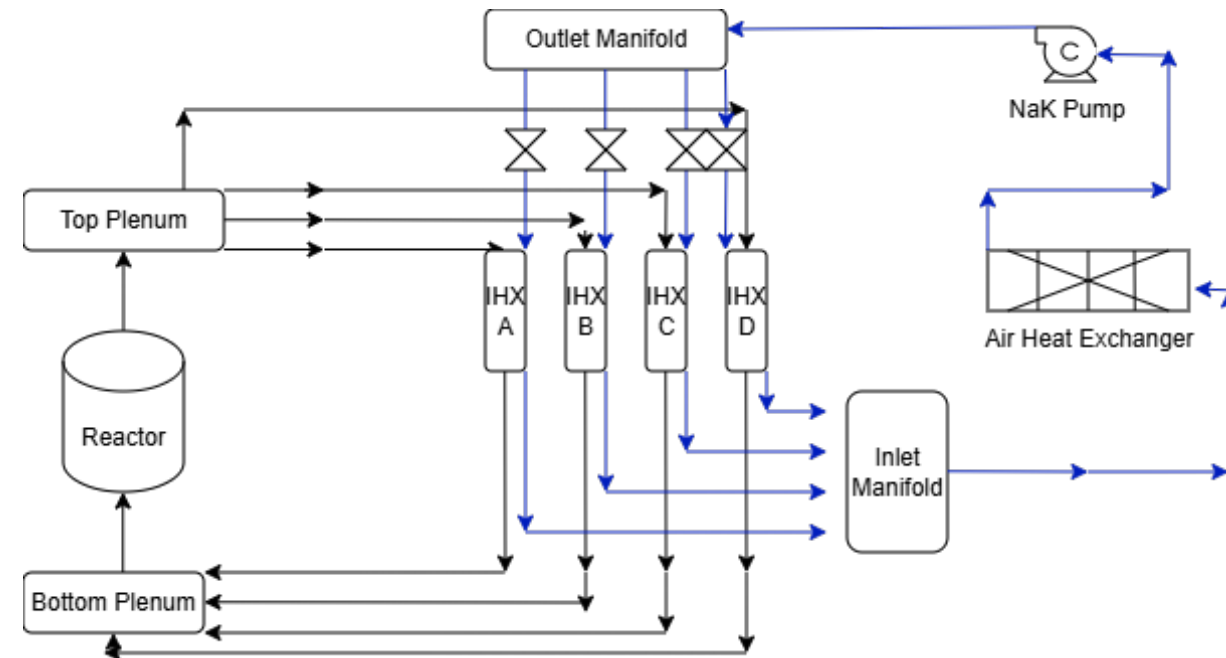
Transient Analysis Workflow



System Configuration

- Hydraulic resistance of each secondary loop is slightly different, and servo valves were used to adjust the resistance to keep flow rates in each loop approximately equal
- Secondary NaK valves can be closed to stop flow to each IHX

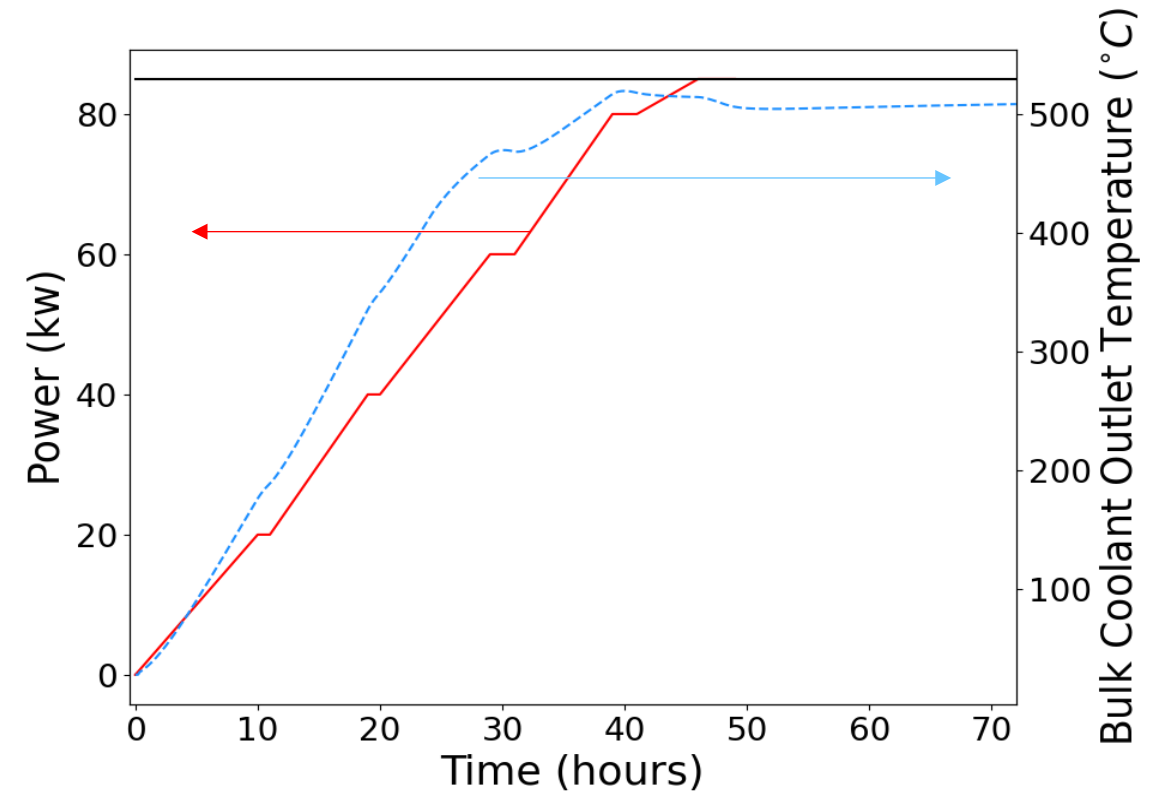
Name	Steady State Value	Transient Value
Peak Fuel Centerline Temperature Limit	900°C	900°C
Peak Cladding Temperature Limit	764°C	764°C
Maximum Bulk Coolant Temperature Limit	530°C	704°C (650°C for anticipated events)



Startup Transient

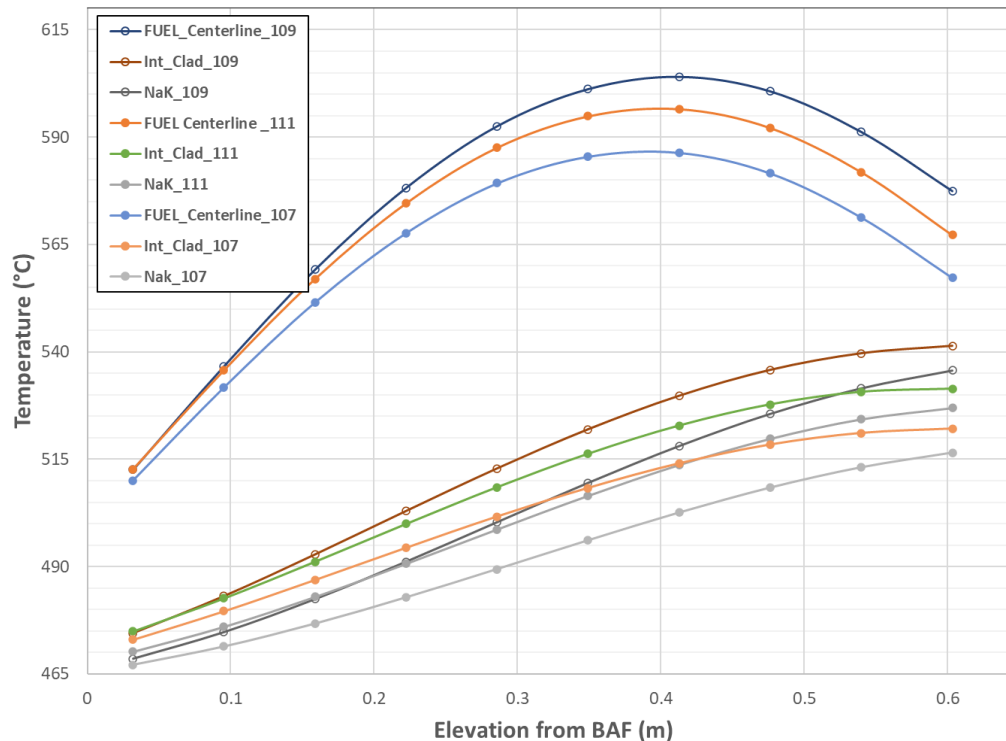
- Power is ramped from 0 to 100% linearly over 48 hours, with a few holds
- Startup can create challenging thermomechanical stresses, especially in the top plenum
- Event Sequence:

Time (hh:mm:ss)	Condition
00:00:00	Secondary NaK flowing at 1 kg/s, Air Heat exchanger at 0.09 kg/s. Reactor power at 0 kW
00:00:00	Reactor power ramp begins
24:00:00	Ramp Air Heat Exchanger flow from 0.09 to 0.17 kg/s linearly begins Ramp NaK flow from 1 to 2 kg/s begins
48:00:00	Reactor at full power NaK flow at 2 kg/s Air Heat Exchanger flow at 0.17 kg/s
72:00:00	End of startup transient



Normal Operations

- Conservative hot channel factors are used to determine the hot channel peak fuel and cladding temperatures

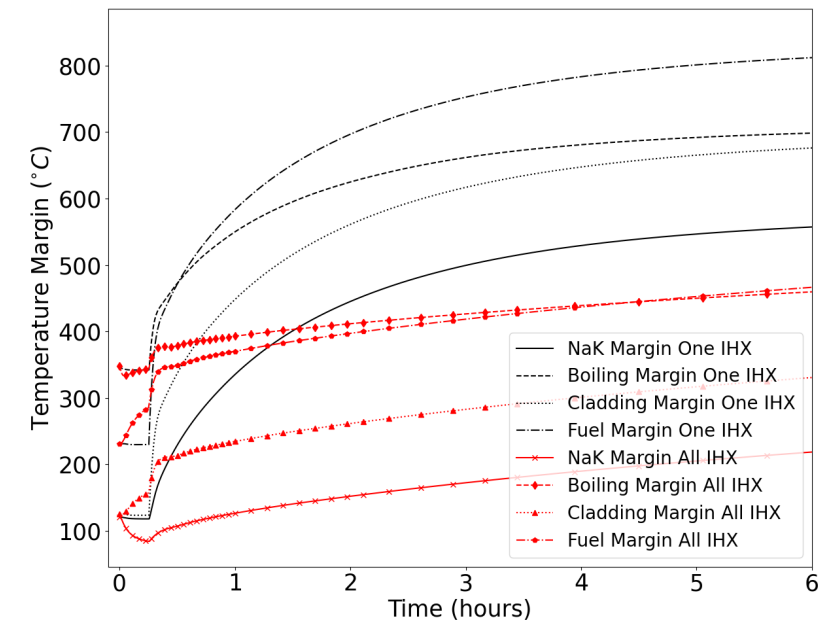
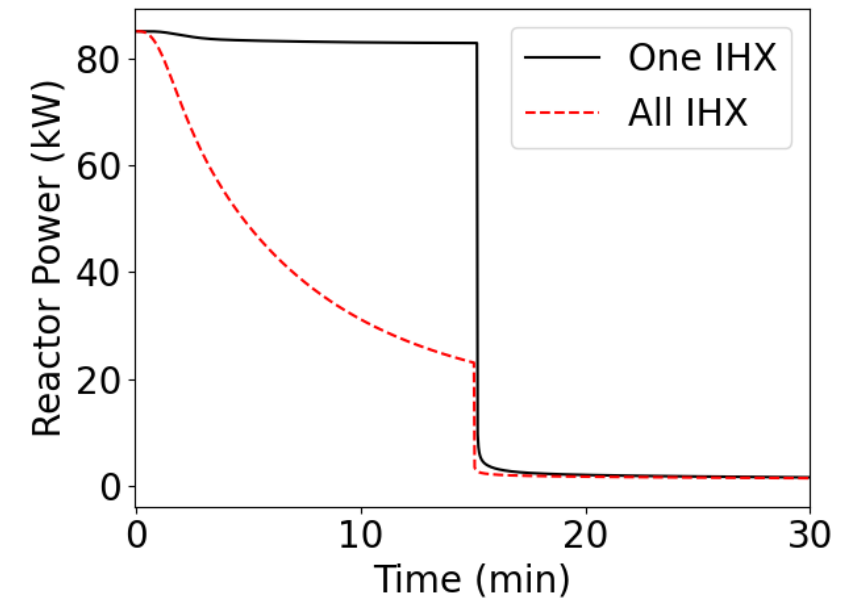


Parameter (Units)	Value
Reactor Power (kW)	85
Primary Coolant Flow Rate (kg/s)	1.53
Secondary Coolant Flow Rate per IHX (kg/s)	0.5
Best-estimate Peak Fuel Temperature (°C)	605
Best-estimate Peak Cladding Temperature (°C)	539
Hot Channel Peak Fuel Temperature (°C)	669
Hot Channel Peak Cladding Temperature (°C)	638
Bulk Coolant Outlet Temperature (°C)	529
Heat Transferred to Secondary System (kW)	75.4
IHX Outlet Temperature (°C)	446

Loss of Heat Sink Accidents

- As an anticipated transient, SCRAM is supposed to occur within 5 minutes, but we allow an extra 10 minutes to ensure this analysis is bounding
- The loss of all IHX is the bounding transient, but loss of one IHX will create asymmetries that can induce stresses

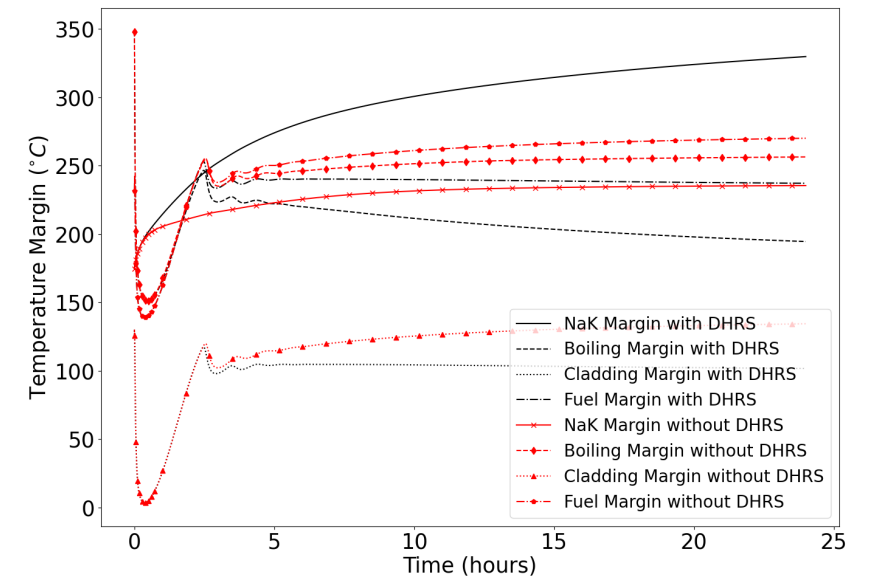
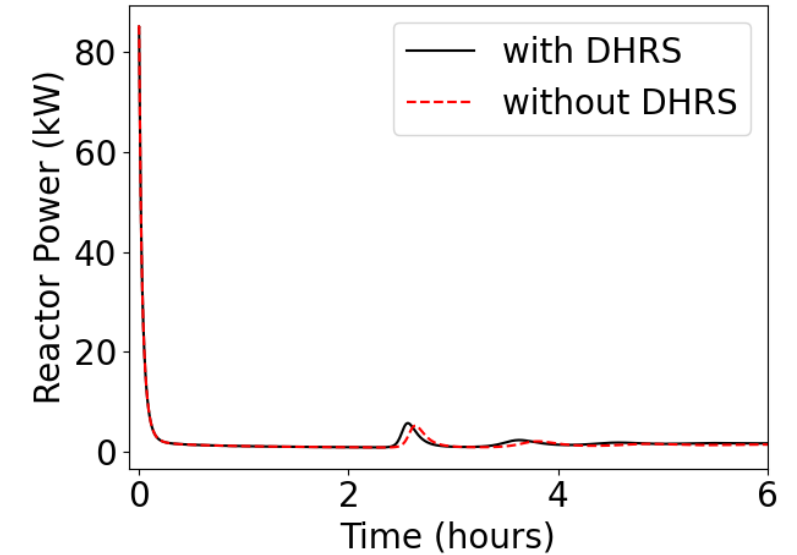
Time (hh:mm:ss)	Condition
00:00:00	Full power steady state
00:00:01	IHX Loop A valve closes instantaneously/All IHX valves close instantaneously
00:15:00	Manual SCRAM Occurs
24:00:00	End of transient



Loss of Flow Accidents

- There is no anticipated pathway for this transient to occur, but we analyze it to ensure survivability
- Without decay heat removal system (DHRS), this is an extremely unlikely event
- Minimum cladding margin is 3.6 K

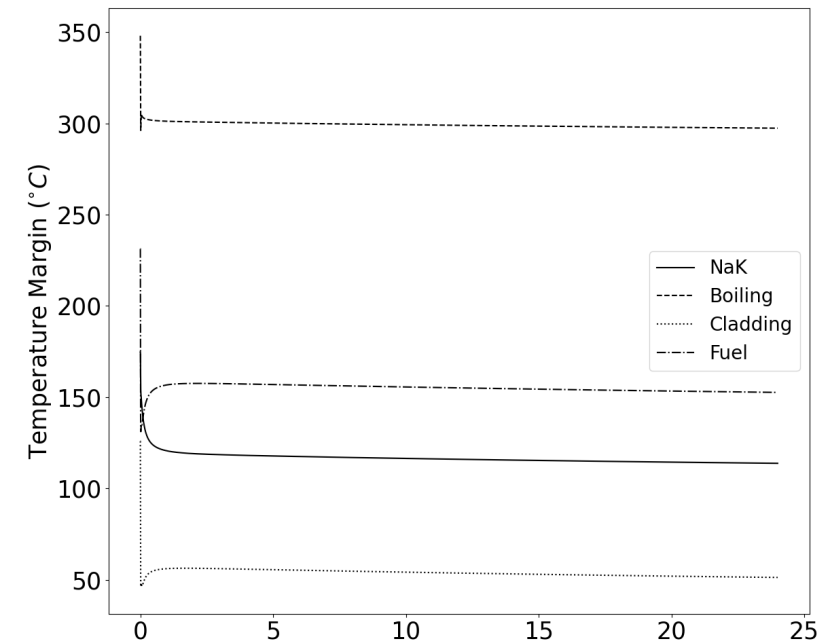
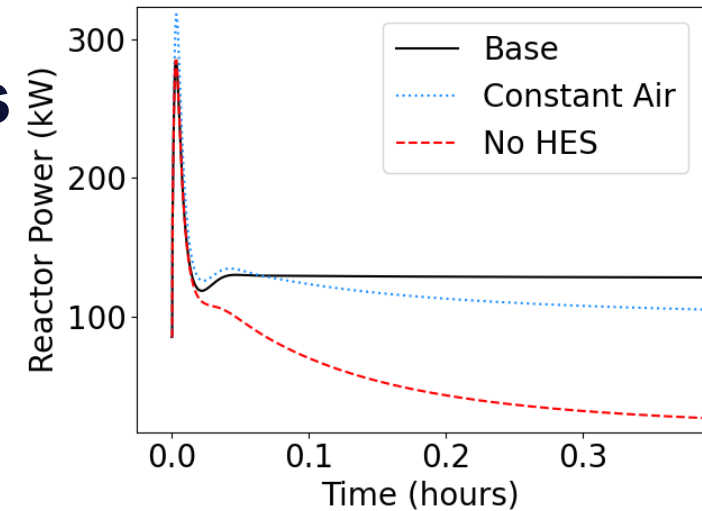
Time (hh:mm:ss)	Condition
00:00:00	Full power steady state
00:00:01	Valves in PCS close to terminate primary flow. Valves in HES close to terminate secondary flow
00:22:24	Minimum cladding temperature margin
01:26:56	First recriticality
24:00:00	End of transient



Unprotected Transient Overpower Accidents

- With heat extraction system active, this is considered an unlikely event
 - Constant air HX speed
 - Variable air HX speed (Base)
- Without heat extraction system active, this is considered an extremely unlikely event
- 0.40\$ inserted over 0.1 seconds
- Minimum cladding margin is 46°C and occurs for constant air heat exchanger flow
- For No HES scenario, valves close 1 second after reactivity insertion starts

Case	Asymptotic Power (kW)
Base	131
Constant Air	97
No HES	11



Conclusions

- Preliminary analysis shows these transients are all survivable from a nuclear safety standpoint
- Ongoing analysis in support of full-power operations uses temperature and flow conditions predicted by RELAP5-3D to inform CFD analysis
- CFD analysis will inform thermomechanical analysis

